

Progress in modeling the impact of ICRF heating on high-Z impurity transport and turbulence stabilization

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- Modeling wave propagation & absorption
- 2 Impact of ICRF heating on the radial high-Z impurity transport
 - Neoclassical: T_i gradient and poloidal density asymmetries
 - Turbulence: affecting the electron to ion heat flux ratio
- 3 Effects of ICRF-accelerated fast ions on turbulence
 - ITG stabilization and T_i steepening in AUG discharge
 - Application to ITER

4 Validation (outlook)

5 Summary

Outline





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Cyclotron Absorption

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ICRF & energetic ions

ICRF heating can generate fast tails up to MeV energies (... stepwise).

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JET: γ -ray imaging(Kiptily, NF 2005)



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Consequences:

Accumulation of trapped ions with their *banana* tips on the Ω_{ic} resonance:

• Poloidally asymmetric density of the resonant ions \implies parallel pressure balance imposes a poloidal electric field pushing ions towards the HFS.



• The critical (crossover) energy, $E_{\rm crt}$, above which the slowing down of energetic particles is mainly on electrons depend on the plasma composition

$$\begin{split} \frac{E_{\rm crt}}{T_e} &\simeq 14.8 \; A_m \; \left(\sum_i \frac{Z_i^2}{A_i} \; \frac{n_i}{n_e} \right)^{2/3} \\ A_m &= M_m/m_p, \qquad q_m = Z_m \; e \end{split}$$

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Consequence:

Even when most of the ICRF power is directly absorbed by ions,

 a significant fraction of the total absorbed ICRF power is collisionally transferred to electrons (higher in the case of H than of ³He minority).





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$$\vec{\nabla} \times \vec{\nabla} \times \vec{E} = \frac{\omega^2}{c^2} \left(\vec{E} + \frac{4\pi i}{\omega} (\mathbf{\tilde{j}_{RF}} + \mathbf{\tilde{j}_{ant}}) \right) \quad \& \quad \text{b.c.}$$



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- Module for fusion alpha particles in DT plasmas (ITER and DEMO). (Bilato, JoP 2014; Brambilla, NF 2015)
- Antenna model for realistic spectrum of the antenna currents \implies estimate of the coupling resistance (scan in n_{φ}).



SSFPQL: 2d Solver for the Steady-State Solution

$$\left(\frac{\partial F_i}{\partial t}\right)_{\rm QL} := \mathcal{Q}_{\rm RF}(F_i) + \mathcal{C}_{\rm coll}(F_i) + S_{\rm NBI} + L_{\rm FIL} \dots = 0$$

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 (0π)



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• Synthetic diagnostics for the neutron rate from fusion reactions, D+D but also with Be, crucial for ITER in the non-activated phase.

(Bilato, NF 2011; Polevoi, IAEA 2019)





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- Antenna module gives the weight of each toroidal component n_φ.

TORIC-SSFPQL Loop: an Example



(5% of H in D plasma and $P_{icrf} = 4$ MW)







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ASDEX Upgrade

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Without NBI




TORIC-SSFPQL Loop: an Example



With D-NBI

(5% of H in D plasma and $P_{icrf} = 4$ MW)

Without NBI



- ICRF-induced deviations of the dist. fun. from the Maxwellian modify both the radial profile and the amplitude of the ICRF absorbed power.
- NBI species resonating with the waves can substantially modify the distribution function and thus the ICRF absorption pattern.



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- by acting on the neoclassical contribution;
- by influencing *turbulence* which counteracts the inward NC convection.



Radial Pfirsch-Schlüter flux (high collisionality)

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 - However, the surface-averaged PS flux is $\underline{\varepsilon}$ -smaller than the local value, $\mathcal{O}(q_{sf}^2)$ (because of large compensations!!!).
- Ambipolarity (momentum conservation&quasineutrality): $\Gamma_Z \approx -\Gamma_i/Z$, the main-ion PS flux is balanced by the high-Z radial flux.



High-Z PS flux

$$\frac{R \left\langle \mathbf{\Gamma}_{\mathbf{Z}}^{\text{neo}} \cdot \nabla r \right\rangle}{\left\langle n_{\mathbf{Z}} \right\rangle} \approx D_{\text{PS,i}} \mathbf{Z} \left[\left(\frac{\mathrm{d} \ln p_i}{\mathrm{d}r} - \frac{T_Z}{Z T_i} \frac{\mathrm{d} \ln p_Z}{\mathrm{d}r} \right) - \frac{3}{2} \frac{\mathrm{d} \ln T_i}{\mathrm{d}r} \right] \\ \approx D_{\text{PS,i}} \mathbf{Z} \left[-\frac{R}{L_{n_i}} + \frac{1}{2} \frac{R}{L_{T_i}} \right] = D_{\text{PS,i}} \frac{R}{L_{n_i}} \mathbf{Z} \left(-1 + \frac{\eta_i}{2} \right)$$

with: $D_{\rm PS} = q_{\rm s}^2 \nu_{c,i} \langle \rho_i^{-1} \rangle^{-1}$ **PS diffusion coefficient** of main ions; $L_A = -A^{-1} dA/dr$, and $L_A > 0$ when A peaked in the core; $\langle \Gamma_Z^{\rm neo} \cdot \nabla r \rangle > 0$ when the flux points outwards; $\langle \cdots \rangle$ surface average; terms $\propto R/L_{T_i}$ are known as temperature screening contribution.



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> Null-condition for the PS flux $\langle \Gamma_Z^{\text{neo}} \cdot \nabla r \rangle = 0$ The PS flux is zero when $\eta_i = L_{n_i}/L_{T_i} = 2$: \implies temperature more peaked than density.





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High-Z PS flux in a poloidally inhomogeneous density

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with $f_c \approx 1 - 1.46\sqrt{\epsilon}$ the fraction of circulating ions ($0.18 \leq f_c \leq 1$).



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Rotating plasmas $\delta/\epsilon \gg 1$

Centrifugal forces are particularly strong for high-Z impurities (high masses) $\implies \delta/\epsilon \gg 1$ ($\delta/\epsilon \approx A_z M_{\varphi,p}^2 \lesssim 6$ for W in ASDEX-Upgrade (AUG), $M_{\varphi,p}$ proton Mach number).

(Angioni, PPCF 2014)



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• Out-In $(\delta > 0) \implies$ increases $\eta_i|_{\min}$: Collisionality of main ions with high-Z impurity larger on the outboard \implies outwards main-ion diffusion compensated by an inward high-Z diffusion.



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In-Out ($\delta \leq 0$) asymmetry of n_z is preferable to avoid Z accumulation in the plasma core.



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• η : The ICRF-heated ions contribute to the *temperature screening*,

 $\eta \approx \eta_i + \nu_{ic} \eta_{ic}$

where $\nu_{ic} = n_{ic}/n_e \ll 1$ is the concentration of the ICRF heated ions.

δ/ε: ICRF heating increases the fraction of trapped ion of the resonating species ⇒ outboard accumulation of resonating ions ⇒ parallel pressure balance imposes a poloidal electric field pushing ions towards the HFS ⇒ effective on high-Z impurities.

(Reinke, PPCF 2012; Casson, PPCF 2015)



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Impurity Density Variation due to ICRF and Plasma Rotation

$$\ln\left(\frac{n_Z}{n_{Z,\text{LFS}}}\right) \approx -\frac{Z_Z}{T_i + Z_{\text{eff}}T_e} \frac{n_{\text{ic}} - n_{\text{ic,LFS}}}{n_e} + \dots (\text{ICRF})$$
$$\frac{m_Z V_{\varphi}^2}{2T_i} \left[1 - \frac{Z_Z}{m_Z} \frac{m_{\text{eff}}}{m_Z} \frac{T_e}{T_i + Z_{\text{eff}}T_e}\right] \left(1 - \frac{R_{\text{LFS}}^2}{R^2}\right) \dots (\text{Centrifugal})$$



Impurity Density Variation due to ICRF and Plasma Rotation



Dependence on the Plasma Rotation



The ICRF in-out asymmetry has a noticeable effect if the toroidal plasma velocity, $V_{\varphi},$ is not too large,

$$\begin{split} V_{\varphi} &< 310 \sqrt{\frac{n_{\rm ic}}{n_e}} \frac{Z_Z}{A_Z} \left(1 - \frac{Z_Z}{A_Z}\right)^{-1} T_{i\,\rm [keV]} \ \frac{\rm km}{\rm s} \approx 100 \frac{\rm km}{\rm s} \\ M_{\varphi} &< \sqrt{\frac{n_{\rm ic}}{n_e}} \frac{Z_Z}{A_Z} \left(1 - \frac{Z_Z}{A_Z}\right)^{-1} \end{split}$$

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- barely satisfied in NBI-heated medium-size plasmas;
- more easily satisfied in fusion reactor plasmas.

(Bilato, NF 2014, NF 2017a)

Example: ASDEX-Upgrade #30812@4.6



40

32

 24

16



R. Bilato et al. - EFTC, Ghent 8.10.2019

Example: ASDEX-Upgrade #30812@4.6





NBI+ICRF



TORIC-SSFPQL

The simulated poloidal density of the ICRF-heated species is such to counteract the centrifugal force acting on the high-Z impurities to avoid high-Z accumulation in the core.

(Odstrcil, PPCF 2018)



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Separate Resonances in the Particle and Heat Fluxes: Quasilinear Model



Roughly (e.g. $v_{\parallel}k_{\parallel}$ is neglected) **High-Z Impurity Diffusivity** $\frac{D_{Z,k}^{\text{turb}}}{\rho_s^2 \omega_{Dk}} \propto \int \frac{\hat{\gamma}_k}{\left(\hat{\omega}_k + \tau_Z \mathcal{K}_y \hat{E}_Z\right)^2 + \hat{\gamma}_k^2} e^{-\hat{E}_Z} \sqrt{\hat{E}_Z} \, \mathrm{d}\hat{E}_Z$ Main-Ion Heat Conductivity $\frac{\chi_{ik}^{\text{turb}}}{\rho_s^2 \omega_{Dk}} \propto \int \frac{\hat{\gamma}_k}{\left(\hat{\omega}_k + \tau_i \mathcal{K}_y \hat{E}_Z\right)^2 + \hat{\gamma}_k^2} \hat{E}_i \left(\hat{E}_i - \frac{3}{2}\right) e^{-\hat{E}_i} \sqrt{\hat{E}_i} \, \mathrm{d}\hat{E}_i$

with: $\tau_{\sigma} = T_{\sigma}/(Z_{\sigma}T_{e})$; $\hat{E}_{\sigma} = E/T_{\sigma}$; $\omega_{Dk} = k_{y}\rho_{s}(c_{s}/R)$ the fluid perpendicular drift frequency $(T = T_{e})$; $\mathcal{K}_{y} = -[\cos\theta + (s\theta - \alpha\sin\theta)\sin\theta]$ the bi-normal curvature $(s-\alpha)$; $\hat{\gamma}_{k} = \gamma_{k}/\omega_{Dk}$ and $\hat{\omega}_{k} = \omega_{k}/\omega_{Dk}$: mode- growth rate and frequency.

(Angioni, PoP 2015)

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- The $\hat{\omega}_k$ -shift between $D_{i,k}^{turb}$ and $\chi_{i,k}$ maxima is also consequence of the energy dependence of curvature and ∇B .

This $\hat{\omega}_r$ -separation of the modes resonating in D_Z^{turb} and in $\chi_{i,k}$ makes turbulence to counteract neoclassical inward convection without significantly impacting the main-ion heat flux, and thus the energy confinement performances.



Experimental observations

• Core RF (both EC and IC) heating helps in avoiding W core accumulation.



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To address whether the physical mechanism behind EC and IC is the same, it is fundamental to have RF simulation tools to

 Design the discharges to have similar power deposition profiles of EC and IC;



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Modeling for design&analysis

To address whether the physical mechanism behind EC and IC is the same, it is fundamental to have RF simulation tools to

- Design the discharges to have similar power deposition profiles of EC and IC;
- Quantify Q_e/Q_i for further modeling.

Design of ASDEX-Upgrade experiments with comparable EC and IC heating profiles: TORBEAM code for EC and TORIC-SSFPQL for IC. $P_{\rm NBI}\approx7.5$ MW.



(Angioni, NF 2017a)





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- The no-accumulation state is achieved when $Q_e/Q_i \approx 1$.
- At the same Q_e/Q_i , ICRF heating is more effective, since it acts against the high-Z accumulation **also** by decreasing δ/ϵ and increasing η_i in the NC transp.

(Angioni, NF 2017a)²⁹

Gyrokinetic simulations with GKW code show that both W diffusion and W convection due to turbulence increase when $Q_e/Q_i \approx 1$.



Eigenvalues solver of GKW code shows that in the region $Q_e/Q_i \approx 1$

- Turbulent W-diffusion is dominated by dominant ITG modes;
- Turbulent W-convection is dominated by sub-dominant TEM modes

(Angioni, PoP 2015, NF 2017a, NF 2017b)



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Stabilizing – Fast ions

Electrostatic:

- Dilute the thermal ITG drive;
- Resonate with ITG.

Electromagnetic :

- Geometrically stabilize via Shafranov shift;
- Mediate nonlinear energy transferred to high-frequency modes.

Destabilizing – Fast ions

Electromagnetic :

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(Di Siena, IAEA 2019)



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Approximating the distribution function with a Maxwellian (n_f, T_f)

Contribution of a minority species to the growth rate

$$\hat{\gamma}_f \propto \mathbf{n_f} \, \tau_f \, \int \, \frac{\hat{\gamma}_k}{\left(\hat{\omega}_k + \tau_f \mathcal{K}_y \hat{E}_f\right)^2 + \hat{\gamma}_k^2} \left[1 + \eta_f \left(\hat{E}_f - \frac{3}{2}\right)\right] e^{-\hat{E}_f} \, \hat{E}_f^{3/2} \, \mathrm{d}\hat{E}_f$$

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- When $\eta_f > 0$, resonating ions stabilize $(\hat{\gamma}_v < 0)$ if

$$0 \lesssim \hat{E}_{\rm f,res} \lesssim \hat{E}_{\rm f,crt}, \quad \text{with}: \quad \hat{E}_{f,\rm crt} = \frac{3}{2} - \frac{1}{\eta_f}$$



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• Typically $\eta_{\rm NBI} \ll 1$ and $\hat{E}_{\rm f,crt} \ll 1 \Longrightarrow$ negligible stabilization

(Di Siena, NF 2018, PoP 2019)











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 - Stabilization of ITGs, stronger for $\hat{\omega}_k \gg 1$.
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 - Existence of an optimal range of $\tau_f = T_f/(Z_f T_e)$.

ICRF ($\eta_f \gg 1$) via this mechanism might reduce the turbulent main-ion heat and particle fluxes, without substantially reducing the subdominant TEMs and low- ω_r ITGs, important for flushing the high-Z ions.

Impact of a Minority-Fast-Ion Species on the micro-instabilities: GENE



ITG growth rates

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Qualitativaly:

the QL model captures the dependence on η_f , τ_f and $k_y \rho_i$, shading light on the role of this resonant mechanism in the linear GENE simulations.



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ICRF Stabilization in AUG Discharges: ASTRA

- ICRF ³He-minority in D plasmas: this mechanism can explain 40% of the excess of GENE predictions of Q_i w.r.t. experimental estimates. (Bonanomi, NF 2018)
- ICRF H-minority in D plasmas: recent experiments on AUG shows a steepening of T_i in correspondence of the region of maximum η_f and T_f , calculated by TORIC-SSFPQL:



⁽Fable, PPCF 2013; Staebler NF 2017)













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- Linear GENE results show that around the largest η_f ITGs are strongly stabilized ($\rho_{tor} \approx 0.25$).
- Preliminary nonlinear global GENE results \implies reduction of the main ion heat flux, Q_i .



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ICRF Scheme in ITER at Full-Field



• $B_0 = 5.2$ Tesla, $f_{ic} = 51$ MHz, D-T(³He) ICRF scheme: in the intial phase ($T \leq 10$ keV) \implies energetic ICRF-³He tails.



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This stabilization mechanism might help in the initial phase of the discharge.

⁽Di Siena, NF 2018)



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Interconnecting simulation tools for

- ICRF (here: TORIC-SSFPQL)
- Iransport (here: ASTRA(+TGLF))
- Gyrokinetics (here: GENE and GKW)



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- ICRF (here: TORIC-SSFPQL)
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helps to design & analyze the present and future experiments using ICRF to

• impact the high-Z impurity radial transport by



Interconnecting simulation tools for

- ICRF (here: TORIC-SSFPQL)
- 2 Transport (here: ASTRA(+TGLF))
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- impact the high-Z impurity radial transport by
 - reducing the inward neoclassical convection through the
 - increase of the temperature screening contribution $(\eta_i \uparrow)$;
 - decrease of the out-in asymmetry $(\delta/\epsilon\downarrow)$ due to centrifugal forces
 - \implies This effect is limited to slow-rotating plasmas (e.g. ITER).



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- Gyrokinetics (here: GENE and GKW)

- impact the high-Z impurity radial transport by
 - reducing the inward neoclassical convection through the
 - increase of the temperature screening contribution $(\eta_i \uparrow)$;
 - decrease of the out-in asymmetry $(\delta/\epsilon\downarrow)$ due to centrifugal forces
 - \implies This effect is limited to slow-rotating plasmas (e.g. ITER).
 - **increasing** turbulent diffusion and convection of high-Z impurities without substantially increase the heat fluxes of the main ions.



Interconnecting simulation tools for

- ICRF (here: TORIC-SSFPQL)
- 2 Transport (here: ASTRA(+TGLF))
- Gyrokinetics (here: GENE and GKW)

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helps to design & analyze the present and future experiments using ICRF to

- impact the high-Z impurity radial transport by
 - reducing the inward neoclassical convection through the
 - increase of the temperature screening contribution $(\eta_i \uparrow)$;
 - decrease of the out-in asymmetry $(\delta/\epsilon\downarrow)$ due to centrifugal forces
 - \implies This effect is limited to slow-rotating plasmas (e.g. ITER).
 - **increasing** turbulent diffusion and convection of high-Z impurities without substantially increase the heat fluxes of the main ions.
- stabilize ITG turbulence by
 - creating fast ions resonating mainly with ITG modes.

 \Longrightarrow In ITER this mechanism might play a beneficial role in the initial phase of the discharge.

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NF = Nuclear Fusion; PPCF = Plasma Physics and Controlled Fusion; PoP = Physics of Plasmas; JoP = Journal of Physics: Conference Series; <math>CPC = Computer Physics Communications.

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